

# Prospects and alternatives for development of US stellarator program

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on behalf of U.S. stellarator collaborators\*

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# Stellarators: external confinement

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- Goal: steady-state, ignited, disruption-free reactor operation with predictable performance and minimum possible requirements for external control.
- Performance determined by 3-D stellarator field
  - MHD equilibrium and stability
  - Energy, particle, and impurity transport
  - Alpha particle confinement
  - Divertor
- Challenge: develop and validate techniques which allow optimization and confident extrapolation of designs to reactor-scale devices.
- US can use its unique capabilities to lead in this area, if it acts soon.

# Stellarator configuration design requires integrated optimization across multiple domains . . . in 3-D

## Confinement volume:

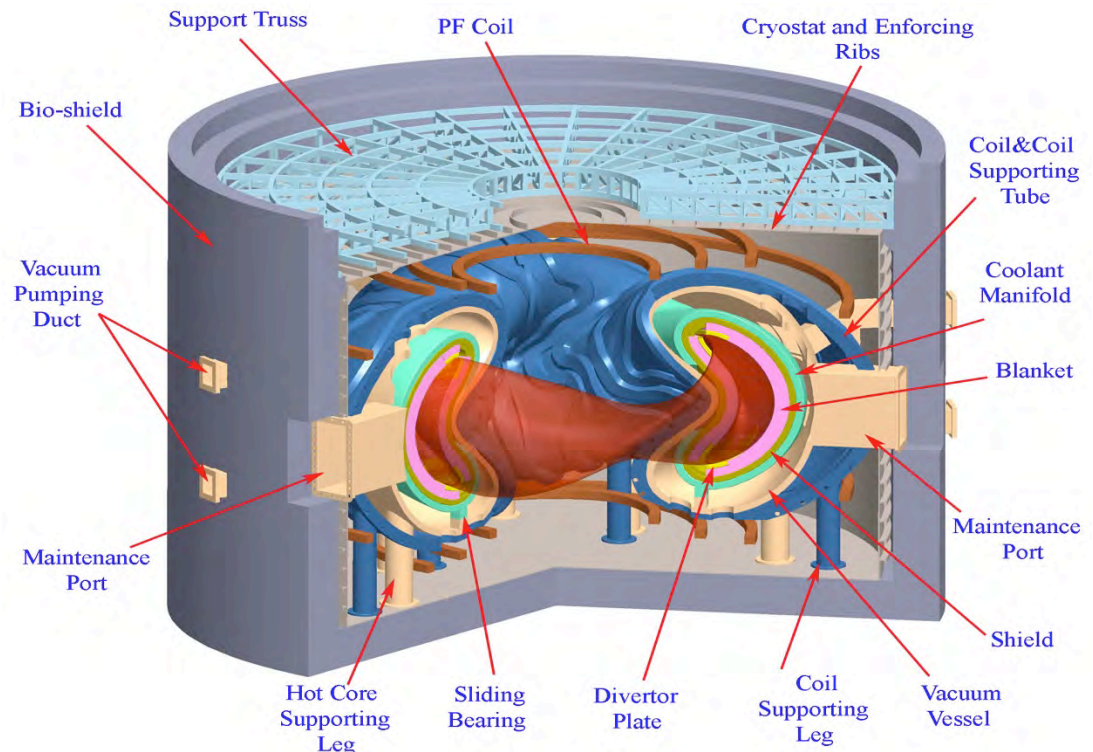
Flux surface quality  
MHD equilibrium, stability  
Neoclassical transport  
Turbulent transport

## Divertor:

Particle/heat exhaust  
Heat flux control  
Impurity control

## Coil/blanket system:

Performance  
Support  
Construction  
Access, maintenance

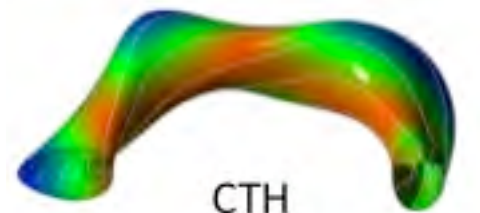
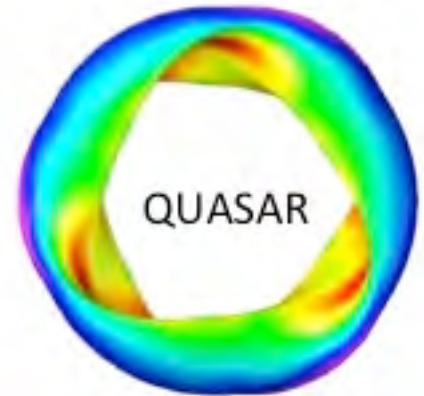


*ARIES-CS reactor concept  
quasi-axisymmetric stellarator  
(based on NCSX/QUASAR design)*

# A US world-leading program in 3-D configurations is critical for finding optimum fusion solutions

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- U.S. design tools optimize plasma/coil configurations by evaluating properties against physics and engineering targets.
- Configurations optimized to reduce neoclassical transport, increase flow shear, achieve high-beta were available by 2007.
- Studies since 2007 have been extended to improve coils, access, and targeted transport processes.
- *Extension of all aspects of stellarator optimization is the defining goal of the US stellarator initiative for the next decade.*
  - *Experimentally test and validate theory.*
  - *Extend understanding and design capabilities.*
  - *Continue to improve designs*



# There is more than one way to optimize a stellarator

## QUASAR (*né* NCSX)

$R = 1.4\text{m}$

$R/a \approx 4.4$

$t_{\text{total}} \sim 0.4\text{-}0.7$

$t_{\text{bootstrap}} \leq 0.15$

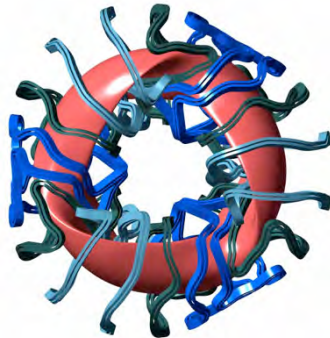
$\beta_{\text{stab}} \geq 6\%$

Helical ripple = 0.1-0.4%

(reduced neoclassical xport)

Quasi-axisymmetry (RS tokamak)

Compact system size



## W7-X

$R = 5.5\text{m}$

$R/a \approx 11$

$t_{\text{total}} \sim 0.83\text{-}1.25$

$t_{\text{bootstrap}} \leq 0.05$

$\beta_{\text{stab}} \geq 5\%$

Helical ripple  $\approx 1\%$

(reduced neoclassical xport)

Isodynamic (Shafranov shift  $\sim 0$ )

Large system size

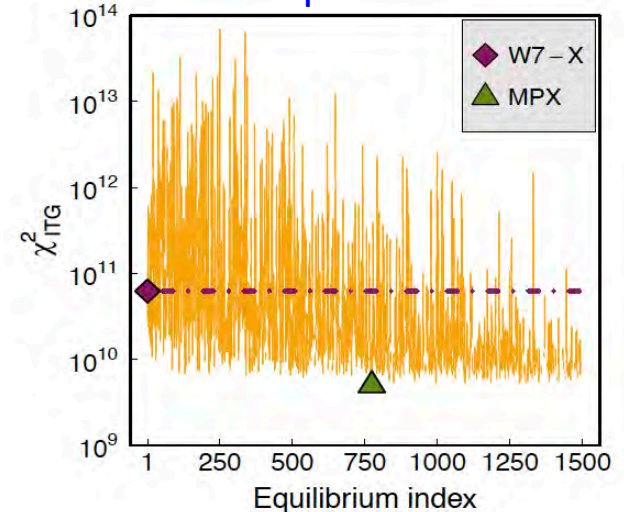


- Which is better??????
- Same physics basis, different design choices
- Depends on actual vs predicted performance  $\Rightarrow$  *experiments!*

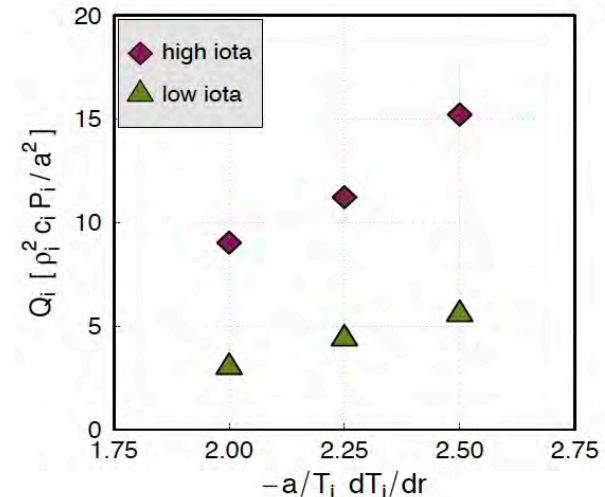
# Stellarator optimization continues even after devices are built

- STELLOPT coupled with GENE turbulence calculations reveals neighbors of W7-X with reduced ITG-driven transport.
- Similar configurations can then be realized in W7-X for experiments by varying coil current ratios.
- These techniques allow anomalous transport mechanisms to be identified in experiments and then reduced by improved design of the external field.
- Similar types of configuration “tweaking” used to improve MHD stability, alpha confinement, etc.

Hunt for improved ITG config.



Realization in actual W7-X





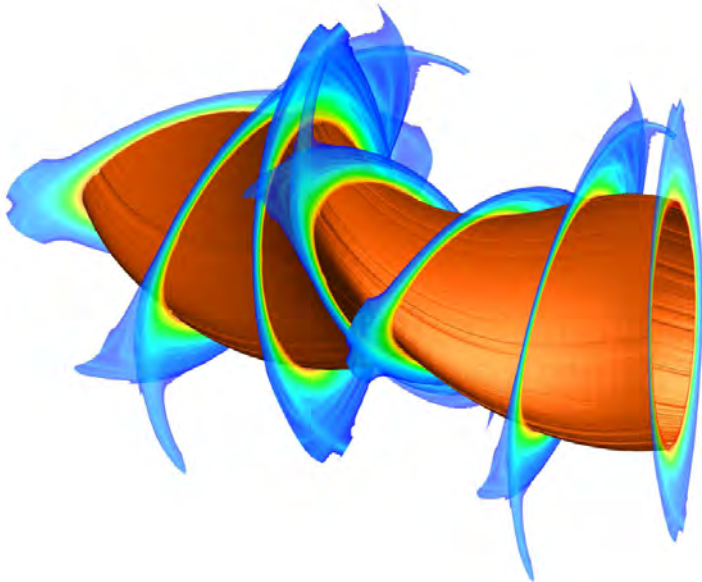
# Divertor design is the next target for optimization

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- Divertors are outside the closed flux surfaces: new methods needed.
- Integration of 3-D plasma and PMI modeling, iterative shaping of divertor plates.
- Experimental validation essential.
- Focus of US activity on W7-X: high heat flux, PMI; coord. w/university exp'ts  
See talk by O. Schmitz
- ⇒ Divertor optimization tools for inclusion in design process

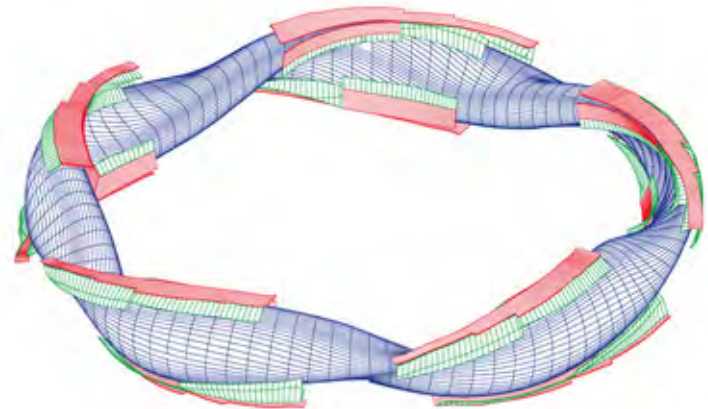
## Large Helical Device (LHD)

Test helical divertor performance with stochastic layer



## Wendelstein 7-X (W7-X)

Test island divertor performance vs. topological stability



# US has near-term opportunities to excel in stellarator optimization and validation

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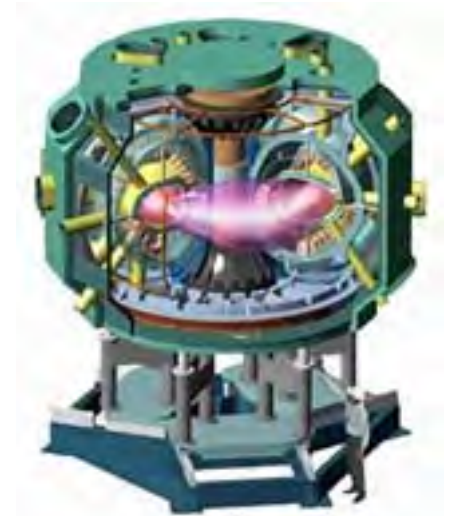
Present US stellarator funding ~\$5-6M/year, but could rapidly expand in steps:

1. **Increased support for 3D theory/computation** [**Increment \$3-4M/yr**] *Talk M. Landreman*
2. **Expand US partnership on W7-X** Immediate expansion to **\$5M/yr and then to \$10-12M/yr** over next 5 years would allow US to increase involvement in divertor and core physics, deploy pellet fueling (ITER prototype) and take advantage of steady-state capabilities. *Talk G. Wurden*
3. **Upgrade 2 existing, add 1 new facilities to explore 3-D divertor physics.** HSX: Heating, flexible divertor; CTH: Increase ohmic heating; HIDRA(WEGA): PMI studies in stellarator environment. [**Increment \$6-7M/yr**] *Talk O. Schmitz*
4. **Mid-size US stellarator experiment.** Fastest implementation (QUASAR, **\$130M construction over 5 yrs, ~\$60M/yr operation**) would use NCSX design and coils. *Next slides.*
5. **Develop design options and mission-need for next-step U.S. stellarator.** Possible missions range from a DD JET-scale device to an FNSF. (**≥ 5 years, ~\$5M/yr**) *Talk: M. Zarnstorff.*



# QUASAR research goals

- Assess/refine 3-D quasi-axisymmetric (QA) configuration
- Unique in world, mimics tokamak symmetry in B-field
- Configuration with  $\sim 25\%$  of transform from plasma current (bootstrap).
- Search for disruption thresholds
- Pressure limits and mechanisms
- Confinement, transport with very low QA ripple
  - Comparison with tokamak
- Role of equilibrium islands, tearing modes, reversed shear
  - 48 trim coils for island production or compensation
- Develop 3-D ergodic divertor configuration
- How much 3-D shaping (external transform) is really needed?
  - Simpler design possible?
  - Continuum with tokamak
- Use configuration flexibility to explore innovations in optimization, e.g. reduction of anomalous transport



$$B = 2 \text{ T}$$

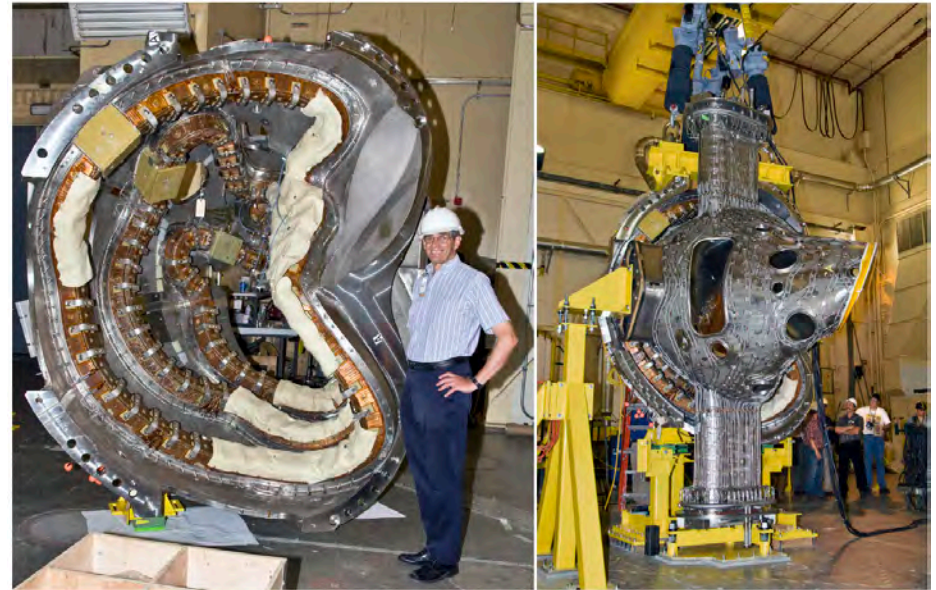
$$\Delta t = 0.5 \text{ (2T)} - 2 \text{ s (1.25T)}$$

$$P_{\text{NBI}} = 6 \text{ MW}$$

$$P_{\text{RF}} = 6 \text{ MW (upgrade)}$$

# Possible QUASAR implementation

- Scoping study (1 year, \$5M)
  - Use NCSX design and coils
    - Refine unbuilt components
      - Trim coils, etc
  - Update physics basis
    - Mission enhancements
    - Partnerships
      - ORNL, MIT, universities, . . .
      - China, others
  - Update cost, schedule estimates
    - Construction: ~\$130M, 4-5 years (China: offer to contribute 1/3)
    - Operation: \$60M/yr
- QUASAR could deliver data needed for optimization within the decade.
- Design of follow-on experiments should proceed in parallel and incorporate emerging improvements.



# Conclusions

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- Integrated design optimization is essential to developing the stellarator as an steady-state, ignited, disruption-free reactor candidate.
- Powerful numerical optimization techniques for complex 3-D fields have been developed, but need experimental validation, and new optimization challenges (e.g., divertors) must be met.
- The US has led the development of optimization tools, and has attractive opportunities to extend its leadership in the next 5-10 years by leveraging domestic and international partnerships in experiments and theory/computation. This will require timely increases in support from present low levels.
- Failure to act will reduce the US to the role of a spectator in a field it largely created.